



Fundamentals of Nuclear Engineering

Brent J. Lewis, PhD, PEng, FCNS

Emeritus Professor and Past COG/UNENE/NSERC Industrial Research Chair in Nuclear Fuel Royal Military College of Canada, Kingston, Ontario, Canada

E. Nihan Onder, PhD

Thermalhydraulics Engineer/Analyst and Nuclear Fuel Research Scientist Canadian Nuclear Laboratories Chalk River, Ontario, Canada

Andrew A. Prudil, PhD, PEng

Nuclear Fuel Safety Scientist Canadian Nuclear Laboratories Chalk River, Ontario, Canada



This edition first published 2017 © 2017 John Wiley & Sons Ltd.

All rights reserved. No part of this publication may be reproduced, stored in a retrieval system, or transmitted, in any form or by any means, electronic, mechanical, photocopying, recording or otherwise, except as permitted by law. Advice on how to obtain permission to reuse material from this title is available at http://www.wiley.com/go/permissions.

The right of Brent J. Lewis, E. Nihan Onder and Andrew A. Prudil to be identified as the authors of this work has been asserted in accordance with law.

Registered Offices

John Wiley & Sons, Inc., 111 River Street, Hoboken, NJ 07030, USA John Wiley & Sons Ltd, The Atrium, Southern Gate, Chichester, West Sussex, PO19 8SQ, UK

Editorial Office

The Atrium, Southern Gate, Chichester, West Sussex, PO19 8SQ, UK

For details of our global editorial offices, customer services, and more information about Wiley products visit us at www.wiley.com.

Wiley also publishes its books in a variety of electronic formats and by print-on-demand. Some content that appears in standard print versions of this book may not be available in other formats.

Limit of Liability/Disclaimer of Warranty

While the publisher and authors have used their best efforts in preparing this work, they make no representations or warranties with respect to the accuracy or completeness of the contents of this work and specifically disclaim all warranties, including without limitation any implied warranties of merchantability or fitness for a particular purpose. No warranty may be created or extended by sales representatives, written sales materials or promotional statements for this work. The fact that an organization, website, or product is referred to in this work as a citation and/or potential source of further information does not mean that the publisher and authors endorse the information or services the organization, website, or product may provide or recommendations it may make. This work is sold with the understanding that the publisher is not engaged in rendering professional services. The advice and strategies contained herein may not be suitable for your situation. You should consult with a specialist where appropriate. Further, readers should be aware that websites listed in this work may have changed or disappeared between when this work was written and when it is read. Neither the publisher nor authors shall be liable for any loss of profit or any other commercial damages, including but not limited to special, incidental, consequential, or other damages.

Library of Congress Cataloging-in-Publication Data

Names: Lewis, Brent J., 1955- author. | Onder, E. Nihan, 1972- author. | Prudil, Andrew A., 1987- author.

Title: Fundamentals of nuclear engineering / Brent J. Lewis, PhD, PEng, FCNS, Emeritus Professor and Past COG/UNENE/NSERC Industrial Research Chair in Nuclear Fuel, Royal Military College of Canada, Kingston, Ontario, Canada, E. Nihan Onder, PhD, Thermalhydraulics Engineer/Analyst and Nuclear Fuel Research Scientist, Canadian Nuclear Laboratories, Chalk River, Ontario, Canada, Andrew A. Prudil, PhD, PEng, Nuclear Fuel Safety Scientist, Canadian Nuclear Laboratories, Chalk River, Ontario, Canada.

Description: Chichester, West Sussex, United Kingdom: John Wiley & Sons, Inc., [2017] | Includes bibliographical references and index.

Identifiers: LCCN 2016039544 (print) | LCCN 2016048630 (ebook) | ISBN 9781119271499 (cloth) | ISBN 9781119271543 (pdf) | ISBN 9781119271550 (epub)

Subjects: LCSH: Nuclear engineering.

Classification: LCC TK9145 .L49 2017 (print) | LCC TK9145 (ebook) | DDC 621.48--dc23

LC record available at https://lccn.loc.gov/2016039544

Hardback ISBN: 9781119271499

Cover image: © Martin Turner/Gettyimages Cover design by Wiley

Set in 10/12pt Warnock by SPi Global, Chennai, India

10 9 8 7 6 5 4 3 2 1

Contents

	Preface xv		
	Acknowledgements xvii		
	About the Companion Website xix		
	Prologue 1		
	Introduction 1		
	Organization of the book 2		
1	Atomic and Nuclear Theory 9		
1.1	Historical Review 9		
1.2	Models of the Nucleus 10		
1.2.1	Field Unification Theories 10		
1.2.2	Nuclear Composition and Stability 13		
1.2.3	Mass-energy Relationships 17		
1.2.4	Decay Schemes 20		
1.2.5	Nuclear Models 23		
1.2.5.1	Nuclear Shell Model 24		
1.2.5.2	Parity 26		
1.2.5.3	Extreme Single Particle Shell Model 27		
1.2.6	Nuclear Transformations 29		
1.2.6.1	Gamma Transitions 30		
1.2.6.2	Beta Transitions 32		
1.2.6.3	Alpha Decay 35		
1.2.7	The Fission Process 38		
1.2.7.1	<u> </u>		
1.2.7.2			
1.2.8	Nuclear Reactions 41		
1.2.9	Cross-sections and Neutron Activation Reactions 42		
1.2.9.1	Rates of Reaction 45		
1.2.9.2	e		
	Bibliography 48		
	Exercises 48		
	Problems 49		
2	Nuclear Reactor Design and Physics 53		
2.1	Overall Concept and Description of Nuclear Reactors 53		
2.1.1	Brief Summary of Types of Reactor 54		

vi	Contents	
I		
		Light Water Reactors 56
	2.1.1.2	Heavy Water Reactors 59
	2.1.2	Small Nuclear Power Reactors 60
	2.1.3	Advanced Nuclear Power Reactors 62
	2.1.4	Generation IV Designs 65
	2.2	Neutron Diffusion 71
	2.3	Slowing Down of Neutrons 75
	2.3.0.1	Slowing Down of Neutrons in an Infinite (No Leakage) Non-absorbing (No Fuel)
	2202	Media 79
	2.3.0.2	Slowing Down in Infinite (No Leakage) Media with Capture 81
	2.3.1 2.3.2	Fermi Age Equation 81
		Solution of the Fermi Age Equation 82
	2.4	Criticality and the Steady State 86
	2.4.1	The Critical Equation 90
	2.4.1.1 2.4.1.2	One-group Critical Equation 90
	2.4.1.2	Modified One-group Critical Equation 91 Two-group Critical Equation 91
	2.4.1.3	Fermi Age Critical Equation 92
	2.4.1.5	
	2.4.1.6	Calculation of Buckling 93 Calculation of Critical Size and Composition 93
	2.4.1.0	Reflector 94
	2.4.2.1	One-group Diffusion Method 95
	2.5	Advanced Reactor Physics 96
	2.5.1	Derivation of the Neutron Transport Equation 97
		Neutron Scattering 98
		Neutron Losses Due to Absorption 99
	2.5.1.2	Neutron Production from Fission 99
	2.5.1.4	Loss of Neutrons by Streaming from the Volume 99
	2.5.1.5	Other Neutron Sources 100
	2.5.1.6	Combining to Obtain the Neutron Transport Equation 100
	2.5.2	Approximations to the Transport Equation 101
	2.5.2.1	Time Independent Formulation 101
	2.5.2.2	Multiple Energy Group Approximation 101
	2.5.2.3	Neutron Diffusion Equation 102
	2.5.3	Solution of the Transport and Diffusion Equations 106
	2.5.3.1	Deterministic Methods 106
	2.5.3.2	Cell and Lattice Method 106
	2.5.3.3	Single-group Neutron Diffusion Equation 107
	2.5.3.4	Multigroup Neutron Diffusion Equation 107
	2.5.3.5	Monte-Carlo Methods 109
	2.5.3.6	Perturbation Theory and the Adjoint Flux 110
		Bibliography 114

${\bf Nuclear\ Reactor\ Dynamics\ and\ Control} \quad 120$ 3 Overview of Reactor Kinetics Behaviour 120 3.1

3.1.0.1 Prompt Critical State *124* 3.1.0.2 Prompt Jump (or Drop) *124*

Exercises 115 Problems 115

3.2	Point Reactor Model and the Inhour Equation 125			
3.2.0.1	Inhour Equation 128			
3.2.0.2				
3.2.0.3	Prompt Jump Approximation 131			
3.2.0.4	Dynamics of Space–Time Kinetic Equations 132			
3.2.1	Reactivity Effects and Poisoning 135			
3.2.1.1	Effect of Temperature and Voids on Reactivity 136			
3.2.1.2				
3.2.1.3				
	Reactor Control 138			
	Control Rods 140			
	Cluster and Cruciform Rods 141			
	Partially-inserted Control Rods 144			
	Zone Controllers 144			
	Chemical Shim and Burnable Poisons 144			
	Burnable Poisons 146			
	Nuclear Fuel Management 146			
	Isotopic Composition of Fuel 146			
3.4.1	Isotope Concentrations and Depletion Equations 147			
3.4.1.1				
3.4.1.2	High-purity Isotopic Production (²³⁹ Pu) 152			
3.4.2	Production of Fission Products 154			
3.4.3	Effect of Burnup on Reactivity 156			
3.4.3.1	Fission Product Effect on Reactivity 156			
3.4.3.2	Xenon (Xe-135) Poisoning 157			
3.4.3.3	Samarium (Sm-149) Poisoning 159			
3.4.3.4	²³⁹ Pu Peak and Fuel Reactivity 160			
3.4.4	Thorium Depletion Equations 162			
3.4.5	Refueling Scheme Optimization 165			
3.4.5.1				
3.4.5.2	On-power Refueling 166			
3.4.5.3	Batch Refueling 168			
	Bibliography 168			
	Exercises 169			
	Problems 169			
4	Nuclear Reactor Materials and Fuel Engineering 174			
4.1	Nuclear Reactor Materials and Fuel Engineering 174 Nuclear Reactor Materials 174			
4.1.1	Fuel Material Properties 186			
4.1.1.1	Metal Fuels 186			
4.1.1.2	Ceramic Fuels: Oxides 188			
4.1.1.3	Ceramic Fuels: Carbides 189			
4.1.1.3	Ceramic Fuels: Nitrides 189			
4.1.1.5				
4.1.1.3	Composite Fuels 189 Structural Material Properties 190			
4.1.3	Structural Material Properties 190 Irradiation Effects on Materials 190			
4.1.3.1	Energy Deposition Causing Radiolysis and Defect Creation 190			
4.1.3.1	Corrosion and Materials Degradation 196			
4.1.4 4.1.4.1	Corresion in Gas-Cooled Systems 196			

4.6.2

Physical State of Fission Products 256

4.1.4.2	Corrosion in Water-Cooled Systems 196
4.1.4.3	Flow-Accelerated Corrosion 198
4.1.4.4	Galvanic Corrosion 198
4.1.4.5	Stress Corrosion Cracking 198
	Crevice Corrosion 199
4.1.4.7	Other Material Degradation Effects 199
4.2	Fuel Production 202
4.2.1	The Fuel Cycle 202
4.2.2	Sources of Reactor Fuel Materials 203
4.2.2.1	Nuclear Fuel Materials 203
4.2.3	Fuel Production 205
	Uranium 205
	Thorium 209
4.2.4	Fuel Fabrication 209
	Uranium Metal 209
	Uranium Dioxide 209
4.2.5	Uranium Isotope Enrichment 212
4.2.5.1	*
4.2.5.2	
4.2.5.3	Laser Methods 217
4.2.6	Reprocessing of Spent Fuel 217
4.2.6.1	Solvent Extraction Method 217
4.3	Fuel Element Thermal Performance 218
4.3.1	Heat Generation 218
	Distribution of Heat Generation in a Reactor Core 221
	Power Peaking Factor 223
4.3.1.3	Heat Generation within a Fuel Element in a Heterogeneous Core (Cylindrical
1,0,1,0	Reactor) 224
4.3.1.4	Heat Generation During Shutdown 225
4.3.1.5	Fission Rate 226
4.3.2	Fuel Material Properties 226
4.3.2.1	Thermal Conductivity Theory 226
4.3.2.2	Effects on Thermal Conductivity 229
4.3.2.3	Correlations of Fuel Thermal Properties 230
4.3.3	Temperature Profiles in Cylindrical Rods 236
4.3.3.1	Fuel Surface Temperature 238
4.4	Fuel Chemistry 240
4.4.1	Phase Diagrams 240
4.4.2	Defect Structures of Oxides 243
4.4.3	Oxygen Potential of the Fuel 244
4.4.4	Fuel Vaporization 246
4.4.4.1	Actinide Redistribution 246
4.5	Fuel Restructuring 248
4.5.1	Pore Migration 248
4.5.2	Columnar-Grain Growth 249
4.5.3	Equiaxed-Grain Growth 250
4.6	Fission Product Behaviour 252
4.6.1	Elemental Yields of Fission Products 253

4.6.2.1	Chemical State of Fission Products 256			
4.6.2.2	Fission Product Migration 263			
4.6.2.3	Effect of Burnup 264			
	Fuel Swelling Due to Solid Fission Products 264			
4.6.3	Swelling Due to Fission Gases 266			
4.6.3.1	Number of Gas Atoms in a Bubble 268			
4.6.3.2	Swelling Due to Gas Bubbles and Overall Gas Balance 271			
4.6.3.3	•			
4.6.3.4				
4.6.4	Fission Gas Release 274			
4.6.4.1	Recoil Release 275			
	Knockout Release 276			
	Diffusional Release 277			
	Gas Release Due to Bubble Interconnection 281			
4.6.4.5				
4.7	Fuel Performance 285			
	Fuel Defect Mechanisms 285			
	CANDU Reactors 285			
	Light Water Reactors 287			
	Power Ramp Failures 288			
	Fuel Performance Codes 290			
	Canadian Codes 290			
	International Codes 290			
4.7.2.2	References 297			
	References 277			
	Evancians 206			
	Exercises 306			
	Exercises 306 Problems 310			
5	Problems 310			
5	Problems 310 Thermal Hydraulics 314			
5.1	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314			
5.1 5.2	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315			
5.1 5.2 5.3	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318			
5.1 5.2 5.3 5.3.1	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321			
5.1 5.2 5.3 5.3.1 5.3.1.1	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1 5.3.2.2	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1 5.3.2.2 5.3.2.3	Problems 310 Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 330			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1 5.3.3.2	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Flow Conservation Equations 336			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1 5.3.3.2 5.4	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 336 Pressure Drop 343			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1 5.3.3.2 5.4 5.4.1	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 336 Pressure Drop 343 Single-Phase Pressure Drop 345			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1 5.3.3.2 5.4 5.4.1 5.4.1.1	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 336 Pressure Drop 343 Single-Phase Pressure Drop 345 Friction Factor 347			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1 5.3.3.2 5.4.1 5.4.1.1 5.4.1.2	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 330 Two-Phase Flow Conservation Equations 336 Pressure Drop 343 Single-Phase Pressure Drop 345 Friction Factor 347 Single-Phase Local Losses 350			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3.1 5.3.3.2 5.4.1 5.4.1.1 5.4.1.2 5.4.2	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 330 Two-Phase Flow Conservation Equations 336 Pressure Drop 343 Single-Phase Pressure Drop 345 Friction Factor 347 Single-Phase Local Losses 350 Two-Phase Pressure Drop 350			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3 5.3.3.1 5.3.3.2 5.4.1 5.4.1.1 5.4.1.2	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 330 Two-Phase Flow Conservation Equations 336 Pressure Drop 343 Single-Phase Pressure Drop 345 Friction Factor 347 Single-Phase Local Losses 350 Two-Phase Pressure Drop 350 Two-Phase Frictional Pressure Drop 353			
5.1 5.2 5.3 5.3.1 5.3.1.1 5.3.1.2 5.3.2.1 5.3.2.2 5.3.2.3 5.3.3.1 5.3.3.2 5.4.1 5.4.1.1 5.4.1.2 5.4.2	Thermal Hydraulics 314 Choice of Coolant 314 Definitions and Simple Two-Phase Flow Relationships 315 Two-Phase Flow 318 Flow Pattern Maps and Transition of Flow Patterns 321 Transition of Flow Patterns in a Vertical Flow 321 Transition of Flow Patterns in a Horizontal Flow 323 Void Fraction 323 Definition and Relationship 323 Flow Pattern Transitions Based on Void Fractions 325 Void Fraction Correlations 325 Single- and Two-Phase Conservation Equations 330 Single-Phase Conservation Equations 330 Two-Phase Flow Conservation Equations 336 Pressure Drop 343 Single-Phase Pressure Drop 345 Friction Factor 347 Single-Phase Local Losses 350 Two-Phase Pressure Drop 350			

x Contents	
-------------------	--

5.5.1	Single-Phase Forced Convective Heat Transfer 396
	Viscous Flow 397
5.5.1.2	Boundary Layer 400
5.5.1.3	Forced Convection Over a Plate (External Flow) 408
5.5.1.4	Forced Convection in a Pipe (Internal Flow) 416
5.5.1.5	Forced Convection in Non-Circular Pipes and Rod Bundles 427
5.5.2	Single-Phase Natural Convective Heat Transfer 442
5.5.2.1	·
5.5.2.2	Empirical Natural Convective Heat Transfer Correlations for Various
0.0.2.2	Geometries 453
5.5.3	Fundamentals of Phase Change and Boiling 456
5.5.3.1	Phase Change 456
5.5.3.2	Classification of Boiling 460
5.5.3.3	Pool and Forced Convective Boiling 460
5.5.4	Subcooled Boiling 463
5.5.4.1	Forced Convection 463
5.5.4.2	Pool Boiling 474
5.5.5	Saturated Nucleate Boiling and Forced Convective Evaporation 474
	Forced Convection 474
	Pool Boiling 488
5.5.6	Critical Heat Flux 490
	CHF Mechanisms for Forced Convective Boiling 494
	Parametric Trends 497
	CHF Prediction Methods for Forced Convective Boiling in Tubes 504
	Separate Effects 516
5.5.6.5	CHF Prediction Methods for Forced Convective Boiling in Rod Bundles 523
5.5.6.6	Pool Boiling CHF 524
5.5.7	Post-Critical Heat Flux 529
	Transition Boiling 530
5.5.7.1	Minimum Film Boiling 531
5.5.7.2	Film Boiling 534
5.5.7.3	Pool Film Boiling 552
5.5.7.4	Condensation 555
	Filmwise Condensation in Vertical Tubes 556
5.5.8.2	Filmwise Condensation in Vertical Tubes 557
5.5.8.3	Dropwise Condensation 559
5.5.8.4	Non-Condensation 339 Non-Condensable Gases 559
5.5.6.4	
	References 559
	Exercises 570
6	Nuclear Reactor Safety 573
6.1	Reactor Licensing and Regulation 573
6.1.1	CANDU Safety Philosophy 573
6.1.2	Licensing Process in the United States 580
6.1.3	Civilian/Military Regulation in Other Countries 585
6.2	General Principles of Reactor Safety 586
6.2.1	Three Levels of Safety 586
6.2.1	Multiple Barriers 587
6.2.3	Reactor Protection System 588
0.4.0	reactor restriction dystem 500

6.3	Engineered Safety Features 590				
6.3.1	Emergency Core Cooling System (ECCS) 590				
6.3.2	Containment Systems 592				
6.3.3	Hydrogen Control 594				
6.4	Reactor Safety Analysis 597				
6.4.1	Loss-of-Coolant Accident (LOCA) 597				
6.4.2	Thermal Hydraulic and Heat Removal Analyses 603				
6.4.2.1	Nuclear Heat Transport 603				
6.4.2.2	Boiling Heat Transfer 611				
6.4.2.3	Blowdown Modelling 613				
6.4.3	Evaluation Computer Codes 618				
6.4.3.1	Example of Codes 619				
6.5	Reliability and Risk Assessment 619				
6.5.1	Risk Determination in Nuclear Plants 622				
6.6	Nuclear Reactor Accidents 624				
6.6.1	Civilian Reactor Accidents 625				
6.6.1.1					
	Chernobyl – Unit 4 629				
6.6.1.2 6.6.1.3	Fukushima Accident 634				
6.6.1.4					
	Activity Release in Accidents 640 Nuclear-Powered Submarine Accidents 640				
6.6.2					
6.7	Radiation Dose Calculations 640				
6.7.1	Radiation Exposure Pathways 640				
6.7.2	Standards of Radiation Protection 642				
6.7.3	Meteorology and the Dispersion of Effluents 645				
6.7.4	Diffusion of Effluents 646				
6.7.4.1	Effect of Deposition/Fallout and Radioactive Decay 652				
6.7.5	Radiation Doses from Radioactive Effluents 653				
6.7.5.1	Whole-Body Dose: External Dose from Plume 653				
6.7.5.2	Internal Inhalation Dose: Thyroid Dose 654				
6.7.5.3	Doses from Ground-Deposited Radionuclides 657				
6.7.5.4	Direct Gamma-Ray Dose 659				
6.8	Nuclear Emergency Response 659				
6.8.1	Accident Classification 660				
6.8.2	Dose Projections 661				
6.8.2.1	Urgent Phase 661				
6.8.2.2	Late Phase 664				
6.9	Fission Product Release and Severe Core Damage Phenomena 669				
6.9.1	Source Term Overview 669				
6.9.2	Fission Product Release 674				
6.9.2.1	Containment Release of Fission Products 679				
6.9.3	Severe Accident Behaviour 686				
6.9.3.1	Severe Core Damage Phenomena 691				
6.9.3.2	Stages of Severe Accident Behaviour 694				
6.9.3.3	Summary of Key Phenomenological Issues 698				
	References 698				
	Exercises 703				
	Problems 706				

7	Health Physics and Radiation Protection 711		
7.1	Interaction of Radiation with Matter 711		
7.1.1	Charged Particles 711		
7.1.1.1	Baryons (Charged Particles: α , p, d, t, 3 He, Fission Fragments) 711		
7.1.1.2	Specific Ionization 713		
7.1.1.3	Fission Products 716		
7.1.1.4	Leptons (Light Charged Particles: Electrons, β^-) 716		
7.1.1.5	Range–Energy Relationships 718		
7.1.2	Electromagnetic Radiation 718		
7.1.2.1	Linear and Mass Attenuation Coefficients 723		
7.1.2.2	Energy Transfer 723		
7.1.2.3	Range 728		
7.1.3	Neutral Particles: Neutrons 728		
7.1.3.1	Elastic and Inelastic Interactions 728		
7.1.3.2	Capture and Spallation Interactions 729		
7.2	Health Physics and Radiation Protection 730		
7.2.1	Doses and Units 730		
7.2.1.1	Irradiation Intensity or Irradiation Rate 731		
	Radiation Other than Photons 732		
7.2.1.3	Biological Factors 732		
7.2.2	Radiological Protection 736		
7.2.2.1	Historical Perspective 737		
7.2.2.2	Protection Quantities 738		
7.2.2.3	Radiation Weighting 738		
7.2.2.4	Operational Quantities 739		
7.2.2.5	Determination of Absorbed Dose Distributions in the Human Body 742		
7.2.2.6	Conversion Coefficients 743		
7.2.2.7	Relationships Between Protection and Operational Quantities 743		
7.2.2.8	Standards of Radiation Protection 743		
7.2.2.9	Estimates of Probability 745		
7.2.2.10			
7.2.3	Dosimetry 750		
7.2.4	Microdosimetry 755		
7.2.4.1	Microdosimetric Quantities and Distributions 758		
7.2.4.2	Calculation of Microdosimetric Spectra 763		
7.3	Biological Effects of Radiation 763		
7.3.1	Biodosimetry 773		
7.3.2	Safety Norms 775		
7.3.2.1	Secondary Norms 776		
7.3.2.1	Irradiation by Contact and Internal Contamination 777		
7.3.2.3	Maximum Allowable Activity in Air (in µCi cm ⁻³) 778		
7.3.2.3	Radiation Protection 779		
7.4.1	Time 782		
7.4.2	Distance 783		
7.4.2	Shielding 783		
7.4.3 7.4.3.1	Plane Circular Source 784		
7.4.3.1	Cylindrical Source 785		
7.5 7.6	Contamination Treatment 788 Space Radiation 788		
7.0	JUANT MANIALIUII /00		

7.6.1 7.6.2	-	304
	Appendix 1	Physical Constants and Conversion Factors 811
	Appendix 2	Table of Atomic Mass Excesses 813
	Appendix 3	Some Values of Nuclear Spins and Parities 900
	Appendix 4	Reactor Physics Parameters 903
	Appendix 5	Physical and Biological Data for Radionuclides 905
	Appendix 6	Cross-Sections of Some Radionuclides 907
	Appendix 7	Properties of Elements and Some Molecules 909
	Appendix 8	Isotopic Cross-Sections 913
	Appendix 9	Direct and Cumulative Thermal Fission Product Yields for Various Fissile Isotopes 926

Index 943

Preface

This book is developed from course lecture notes given in the graduate programme in Nuclear Engineering at the Royal Military College of Canada (RMC), Kingston, ON, and École Polytechnique, Montreal, QC, and an undergraduate course at Queen's University, Kingston, ON. It is further based on subject research in nuclear fuel behaviour, thermal hydraulics and radiation protection (for aircrew and space crew) as research scientists at the Canadian Nuclear Laboratories [formally the Chalk River Laboratories of Atomic Energy of Canada Limited (AECL)] and as a university educator and Industrial Research Chair in Nuclear Fuel sponsored by the CANDU Owners Group (COG), University Network of Excellence in Nuclear Engineering (UNENE) and Natural Sciences and Engineering Research Council (NSERC). This book focuses on undergraduate and graduate-level teaching in nuclear engineering with the development of concepts in a systematic manner. It is relevant to the nuclear professional summarizing some key research developments in the fields of nuclear fuel behaviour, health physics and reactor thermal hydraulics. Moreover, it especially fills an important need and niche as a modern and comprehensive textbook for undergraduate and graduate instruction and the learning of core subjects in atomic and nuclear theory, nuclear reactor physics, nuclear reactor dynamics and control, nuclear fuel engineering, thermal hydraulics, nuclear reactor safety, and health physics and radiation protection. The textbook also contains extensive nuclear and reactor physics data, and fundamental constants detailed in several Appendices as developed from recent data libraries. Solved exercises are provided to augment the learning of the text material. In addition, a number of solved problems used for various tests and examinations in the courses are also included at the end of each chapter. This package therefore provides a complete set of source material and problems with a single textbook for undergraduate and graduate course instruction.

January 2017

Brent J. Lewis E. Nihan Onder Andrew A. Prudil

Acknowledgements

We would like to recognize the use of some source material selected from several nuclear engineering courses. In particular, we would like to acknowledge the use of lecture material derived from the CHE1521S course of Professor R. Jervis at the University of Toronto in the area of nuclear and radiochemistry for Chapter 1 and lecture material from the CHE1531S course of Professor J. Luxat at the University of Toronto on nuclear reactor control systems for Chapter 3. In addition, we are indebted to Dr. Altan Tapucu from École Polytechnique for providing some of his lecture notes from the ENE 6002 and ENE 6107 courses and comments on Chapter 5 and Professor Alberto Teyssedou from École Polytechnique for his contribution to the ENE 6002 and ENE 6107 course materials, and Professor H.W. Bonin of the Royal Military College of Canada for some of his course material and problems on health physics and radiation protection from the CC511 course for Chapter 7. We would like to further acknowledge and thank Professor P. Chan for some lecture material on nuclear fuel design and nuclear safety, Dr. Diana Wilkinson from the Defence Research Development Canada (Ottawa) on radiation biology and Kirsten Avarmaa for her diligent work on the book illustrations.

PowerPoint slides developed from material in this book for courses CC511 (Health Physics and Radiation Protection), CC523 (Nuclear Reactor Engineering) and CC533 (Nuclear Fuel Engineering) were graciously provided by Professors E. Corcoran and P. Chan from the Royal Military College of Canada for use on the Wiley web site.

The book is dedicated to the many graduate students we have had the pleasure to supervise and mentor over the many years. Finally, we would like to thank our colleagues for their continual friendship and collaboration, and especially our families for their patience and loving support.

About the Companion Website

Don't forget to visit the companion web site for this book: www.wiley.com/go/lewisnuc



There you will find valuable material designed to enhance your learning, including:

- Problem solutions
- Teaching slides
- Course outlines

Scan this QR code to visit the companion web site



Prologue

Introduction

Nuclear technology was first developed in the 1940s during research on weapons production during the World War II. Attention turned to commercial nuclear power in the 1950s. Today, nuclear energy is an important source of electricity production for three main reasons: (i) supply, (ii) environmental footprint (i.e., climate change) and (iii) economics.

As shown in Figure P.1, nuclear power provides about 11% of the global electricity needs. In particular, as of 2015, there have been 16 000 reactor years of experience with 436 commercial power reactors in 31 countries that supply 378 000 MW (electrical) of total capacity; in addition, 67 nuclear power reactors are also under construction with 166 reactors being planned (Table P.1). Fifty-six countries operate a total of 240 research reactors as a source of neutrons for scientific research and for the production of medical and industrial isotopes. Moreover, there are about 180 nuclear reactors that power ships and submarines.

From 1990 to 2010, the world electricity (e) capacity rose by 57 GWe (17.75%), with a rise in electricity produced from nuclear power of 755 TWh (40%), as shown in Figure P.2, due to new plant construction (36%), uprating of other plants (7%) and an increase in availability of plants (57%). The USA itself accounts for nearly one third of the world's nuclear electricity (see the first column of Table P.1), where nuclear power plant performance has increased over the past twenty years with capacity factors over 90% in five of the seven years up to 2013. In 2011 and 2012, both capacity and output diminished, with cutbacks in Germany and Japan (i.e., in Japan dropping from 13 TWh in 2010 to 0 TWh in 2015 as seen in Table P.1) following the Fukushima reactor accident (see Chapter 6).

Nuclear power is important because of its relatively low environmental footprint in terms of climate change. The lifecycle greenhouse gas (GHG) emissions from different forms of electricity generation for all phases of the process including construction, operation, and decommissioning are shown in Figure P.3 based on the analysis of twenty studies. This analysis shows that generating electricity from fossil fuels results in much greater emissions than that from nuclear or renewable generation.

Data for costs in the United States for various sources of electricity production from 1995 to 2012 (Figure P.4) show nuclear generation (i.e., for the fuel plus operation and maintenance) at 2.40 cents/kWh, as compared with coal at 3.27 cents/kWh and gas at 3.40 cents/kWh. These costs exclude indirect costs and capital costs that are plant/utility specific and also depend on the age of the plant.

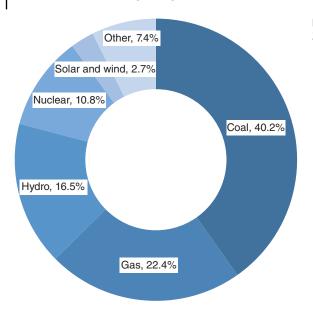


Figure P.1 World electricity production in 2012. Source: World nuclear association.

A finish study for projected electricity costs in 2003 suggested nuclear production at &2.37 cents/kWh, coal at &2.63 cents/kWh and natural gas at &3.22 cents/kWh (Figure P.5). This study assumed a 91% capacity factor, 5% interest rate and 40-year plant life. The relative effects of capital and fuel costs are depicted. Nuclear production specifically has a relatively high capital cost that depends importantly on the financing costs and length of time for construction. On the other hand, the fuel costs are much lower, so that once a nuclear plant is built its costs are more predictable compared to gas or coal. In addition, a carbon tax can impact costs, that is, with carbon emissions trading at &20/t CO₂, the electricity costs for coal and gas increase to &4.25 and 3.92 cents/kWh, respectively. Finally, in 2015, a report from the Institute for Energy Research on the levelized cost of electricity from existing generation resources suggested nuclear production at slightly over \$90/MWh, compared with coal at almost \$100/MWh and gas just over \$70/MWh.

Organization of the book

The book covers a broad range of key areas in the field of nuclear engineering and is organized into seven chapters, consisting of: Chapter 1: Atomic and Nuclear Theory; Chapter 2: Nuclear Reactor Design and Physics; Chapter 3: Nuclear Reactor Dynamics and Control; Chapter 4: Nuclear Reactor Materials and Fuel Engineering; Chapter 5: Thermal Hydraulics; Chapter 6: Nuclear Reactor Safety; and Chapter 7: Health Physics and Radiation Protection. Information in the book is provided at both an introductory and a more advanced level and also draws on, in part, recent state-of-the-art research in nuclear fuel behaviour, reactor safety and thermal hydraulics. The book chapters are presented in a logical manner from basic theory to design, construction, operation, control and safety of nuclear reactors, including the need for health physics and radiation protection. It also contains nine appendices of relevant nuclear and reactor physics data as well as fundamental constants, cross-sections and fission product yields. This work also includes a complete exercise manual with solved problems for the exercises and problems presented at the end of each chapter.

 Table P.1
 World nuclear power reactors and uranium requirements

Country Sillion (whh) Seelectricity No. Mike parces No. Mike gross No. No. Spatial Argantina Spatial No. No. No. Spatial No. No. <th></th> <th>Nuclear electricity generation 2014</th> <th>Reactors operable August 2015</th> <th></th> <th>Reactors under construction August 2015</th> <th>der</th> <th>Reactors planned August 2015</th> <th></th> <th>Reactors proposed August 2015</th> <th></th> <th>Uranium required 2015</th> <th></th>		Nuclear electricity generation 2014	Reactors operable August 2015		Reactors under construction August 2015	der	Reactors planned August 2015		Reactors proposed August 2015		Uranium required 2015	
sh 5.3 4.0 3 1627 1 27 2 1950 2 1300 sh 0 0 0 0 0 0 0 1 1060 2 1300 sh 0 <th>Country</th> <th>Billion kWh (or TWh)</th> <th>% electricity</th> <th>Š</th> <th>MWe net</th> <th>ŏ</th> <th>MWe gross</th> <th>Š</th> <th>MWe gross</th> <th>Š</th> <th>MWe gross</th> <th>tonnes U</th>	Country	Billion kWh (or TWh)	% electricity	Š	MWe net	ŏ	MWe gross	Š	MWe gross	Š	MWe gross	tonnes U
sh 0 0 0 1 1060 sh 0 <td>Argentina</td> <td>5.3</td> <td>4.0</td> <td>3</td> <td>1627</td> <td>1</td> <td>27</td> <td>2</td> <td>1950</td> <td>2</td> <td>1300</td> <td>215</td>	Argentina	5.3	4.0	3	1627	1	27	2	1950	2	1300	215
sh 0	Armenia	2.3	30.7	1	376	0	0	1	1060			88
q 0 0 0 2 288 0 0 2 400 32.1 47.5 4 5443 0	Bangladesh	0	0	0	0	0	0	2	2400	0	0	0
32.1 47.5 7 5943 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 1 45 44 40	Belarus	0	0	0	0	2	2388	0	0	2	2400	0
14.5 2.9 2.9 1.0 14.5 6 0 4 400 15.0 31.8 2 1906 0 0 1 950 0 </td <td>Belgium</td> <td>32.1</td> <td>47.5</td> <td>^</td> <td>5943</td> <td>0</td> <td>0</td> <td>0</td> <td>0</td> <td>0</td> <td>0</td> <td>1017</td>	Belgium	32.1	47.5	^	5943	0	0	0	0	0	0	1017
public 15.0 13.8 2 1906 0 0 1 95.0 4 4000 0	Brazil	14.5	2.9	2	1901	1	1405	0	0	4	4000	326
ps6 16.8 16.8 19 13553 0 0 2 1500 3 3800 public 28.6 0 0 0 0 0 0 4 4400 public 28.6 3.4 26 23144 25 27393 43 49970 136 4400 public 28.6 3.5.8 6 3904 0 0 2 2400 1 400 public 28.6 3.5.8 6 3904 0 0 2 2400 1 400 14.8 3.5 4 2741 1 1700 1 200 0 1 200 4 18.0 6 6 1750 0 0 0 0 0 0 1 100 4 18.0 6 7 1750 0 0 0 0 0 0 0 0 4	Bulgaria	15.0	31.8	2	1906	0	0	1	950	0	0	324
public 36 0 0 0 0 4 4400 public 38.6 3.4 26 23.144 25 27.393 43 49.970 136 54.00 public 38.6 35.8 6 39.44 25 27.393 43 49.970 136 153.00 public 38.6 3.5 6 39.44 1 7.790 2 2400 1 1200 1 1200 1 1200 1 1200 1 1200 1 1200 2 2400 2 2400 2 2400 2 2400 3 2 2400 3 3 3 3 4 4000 3 3 4 4000 3 4 4000 3 4 4000 4 4000 4 4000 4 4000 4 4000 4 4000 4 4000 4 4000 4 4000 4	Canada	98.6	16.8	19	13 553	0	0	2	1500	3	3800	1784
public 8.6 2.4 26 21.44 25 27.393 43 49.970 136 153.000 public 8.6 35.8 6 3904 0 0 2 2400 15 1500 qual 6 36.8 6 3044 0 0 2 2400 1 1200 4 2.6 6 6.1 17 1750 1 1200 1 1500 4 18.8 6 6.1 1750 0 0 0 0 0 0 1 1500 0	Chile	0	0	0	0	0	0	0	0	4	4400	0
public 26. 35.8 6 3904 0 0 2 2400 1 1200 2.0 0 0 0 0 0 0 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 3 2400 3 2 2400 3 2 2400 3	China	123.8	2.4	76	23 144	25	27 393	43	49 970	136	153 000	8161
at 0 0 0 0 0 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 2 2400 3 2400 3 2400 3 2400 3 2400 3 2400 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 3 4 4 1500 3 3 4 4 1500 3 4 4 4 1500 3 4	Czech Republic	28.6	35.8	9	3904	0	0	2	2400	1	1200	999
a. 2.6.6 34.6 4 2741 1 1700 1 1200 1 1500 418.0 76.9 76.9 58 63.130 1 1750 0 0 0 1 1500 a. 14.8 15.8 10.728 0	Egypt	0	0	0	0	0	0	2	2400	2	2400	0
418.0 76.9 58 63130 1 1750 0 0 1 1750 a 91.8 15.8 8 10728 0	Finland	22.6	34.6	4	2741	1	1700	П	1200	1	1500	751
at 15.8 16.728 0	France	418.0	76.9	28	63 130	1	1750	0	0	-	1750	9230
a 14.8 53.6 4 1889 0 0 2 2400 0 0 0 0 0 4 1889 0 4300 2 2400 0 4 4000 a 0 0 0 0 0 0 4 4000 a 1.5 1.5 1 2 2000 2 2000 7 6300 0 0 0 0 0 0 0 0 1 1200 a 0 <td>Germany</td> <td>91.8</td> <td>15.8</td> <td>∞</td> <td>10 728</td> <td>0</td> <td>0</td> <td>0</td> <td>0</td> <td>0</td> <td>0</td> <td>1889</td>	Germany	91.8	15.8	∞	10 728	0	0	0	0	0	0	1889
33.2 3.5 21 5302 6 4300 22 21300 35 40000 0 0 0 0 0 1 30 4 4000 3.7 1.5 1 915 0 0 0 7 6300 0 0 0 0 0 0 0 1 1200 0	Hungary	14.8	53.6	4	1889	0	0	2	2400	0	0	357
0 0 0 0 0 400 3.7 1.5 1 915 0 0 7 6300 0 0 0 0 0 0 1 1200 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 1 0	India	33.2	3.5	21	5302	9	4300	22	21 300	35	40 000	1579
3.7 1.5 1 915 0 0 2 2000 7 6300 0 0 0 0 0 0 0 1 1200 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 1) 0	Indonesia	0	0	0	0	0	0	1	30	4	4000	0
0 0 0 0 0 0 0 1 1200 0	Iran	3.7	1.5	1	915	0	0	2	2000	7	6300	176
0 0	Israel	0	0	0	0	0	0	0	0	1	1200	0
$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	Italy	0	0	0	0	0	0	0	0	0	0	0
0 0 0 0 0 0 2 2000 1 0 0 0 0 0 2 600 2 600 1 0 0 0 0 0 0 1 950 149.2 30.4 24 21677 4 5600 8 11600 0 0	Japan	0	0	43	40 480	33	3036	6	12 947	33	4145	2549
0 0 0 0 0 0 600 2 600 2 600 1) 0 0 0 0 0 0 1 950 149.2 30.4 24 21677 4 5600 8 11600 0 0	Jordan	0	0	0	0	0	0	2	2000			0
1) 0 0 0 0 0 0 1 950 149.2 30.4 24 21677 4 5600 8 11600 0 0	Kazakhstan	0	0	0	0	0	0	2	009	2	009	0
149.2 30.4 24 21677 4 5600 8 11600 0 0	Korea DPR (North)	0	0	0	0	0	0	0	0	П	950	0
	Korea RO (South)	149.2	30.4	24	21 677	4	2600	8	11 600	0	0	5022

Table P.1 (Continued)

	Nuclear electricity generation 2014	Reactors operable August 2015		Reactors under construction August 2015	ler	Reactors planned August 2015		Reactors proposed August 2015		Uranium required 2015	
Country	Billion kWh (or TWh)	% electricity	Ö	MWe net	ŏ	MWe gross	Š	MWe gross	o S	MWe gross	tonnes U
Lithuania	0	0	0	0	0	0	1	1350	0	0	0
Malaysia	0	0	0	0	0	0	0	0	2	2000	0
Mexico	9.3	5.6	2	1600	0	0	0	0	2	2000	270
Netherlands	3.9	4.0	1	485	0	0	0	0	1	1000	103
Pakistan	4.6	4.3	3	725	2	089	2	2300	0	0	101
Poland	0	0	0	0	0	0	9	0009	0	0	0
Romania	10.8	18.5	2	1310	0	0	2	1440	1	655	179
Russia	169.1	18.6	34	25 264	6	2962	31	33 264	18	16 000	4206
Saudi Arabia	0	0	0	0	0	0	0	0	16	17 000	0
Slovakia	14.4	56.8	4	1816	2	942	0	0	1	1200	466
Slovenia	6.1	37.2	1	969	0	0	0	0	1	1000	137
South Africa	14.8	6.2	2	1830	0	0	0	0	8	0096	305
Spain	54.9	20.4	7	7002	0	0	0	0	0	0	1274
Sweden	62.3	41.5	10	9487	0	0	0	0	0	0	1516
Switzerland	26.5	37.9	2	3333	0	0	0	0	3	4000	521
Thailand	0	0	0	0	0	0	0	0	2	2000	0
Turkey	0	0	0	0	0	0	4	4800	4	4500	0
Ukraine	83.1	49.4	15	13 107	0	0	2	1900	11	12 000	2366
UAE	0	0	0	0	3	4200	1	1400	10	14 400	0
United Kingdom	57.9	17.2	16	9373	0	0	4	0899	7	8920	1738
USA	798.6	19.5	66	98 792	2	6018	2	6063	17	26 000	18 692
Vietnam	0	0	0	0	0	0	4	4800	9	0029	0
World	2411	11.5	436	378 995	29	20 107	991	186 704	322	364 920	66 883

Source: World nuclear association.

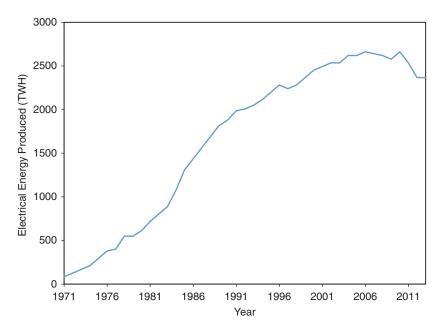


Figure P.2 Nuclear electricity production in the world. Source: World nuclear association.

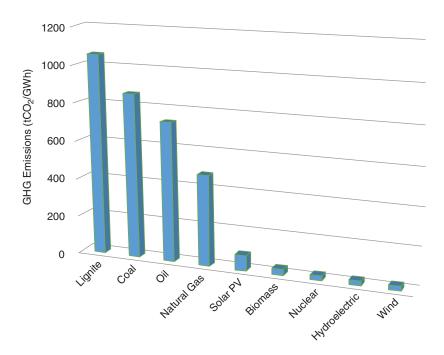


Figure P.3 Average lifecycle greenhouse gas emissions from different sources of electricity generation. Source: World nuclear association.

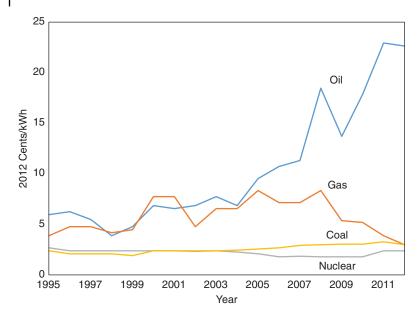


Figure P.4 US electricity production costs from 1995 to 2012. Source: World nuclear association.

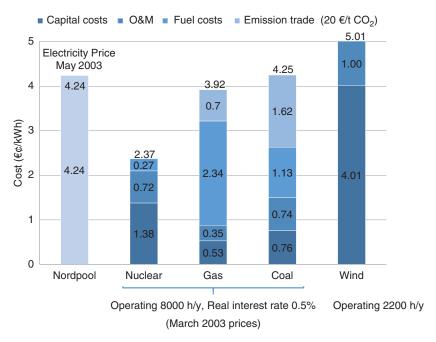


Figure P.5 Projected electricity costs for Finland in 2003. Source: World nuclear association.

Chapter 1 provides an overview of atomic and nuclear theory, including historical developments and a modern understanding of the underlying forces of nature. It particularly explains nuclear composition and stability in nature and how energy release is "thermodynamically" favoured in the fission process. This energy release occurs through a mass loss between the reactants and products as a result of Einstein's mass-energy equivalence relationship and is the source of energy in nuclear power. The chapter also details generalized schemes and systematics for alpha, beta and gamma decay and transition processes. Importantly, by utilizing nuclear models, it explains why specific decay systematics are observed for isotopes in accordance with nuclear selection rules. The theory provides a fundamental understanding of decay processes and their energies as well as the observed nuclear spin and parity of nuclides. It also details the important concepts of nuclear reactions and cross sections as needed for reactor physics analysis in the design of the critical reactor core in Chapter 2.

Chapter 2 illustrates the overall design of reactors, including past, present and future concepts. It describes the physics of criticality, including introductory reactor theory, design and operation. In the steady-state, sources and losses of neutrons in the reactor are examined for the critical state employing diffusion theory for leakage of neutrons from the core. Diffusion theory in itself is an approximation of the more generalized neutron transport equation. The development of this latter equation from advanced reactor theory is also presented, as is its simplification to diffusion theory with the assumption of isotropic neutron scattering. Moreover, the solution of the more detailed neutron transport equation using modern computational methods is explained.

Chapter 3 examines the dynamical nature of the reactor using a "point kinetics" approach. This chapter involves a discussion of the overall reactivity of the system for control of the reactor. The time-dependent behaviour of the reactor for start-up, shutdown or a change in power is discussed as a further extension of the steady-state development in Chapter 2. It details how the reactor can be controlled solely from the production of delayed neutrons that are produced by radioactive decay of some fission products. The effects of the loss of fissile material and the production of fission product "poisons" are discussed. Engineered devices used to control the reactor are further detailed. Also, the specifics for the fuelling of the reactor as well as different fuel management schemes are examined.

Chapter 4 details nuclear reactor materials, including the properties of structural materials, and irradiation effects on materials that can lead to corrosion and materials degradation/deformation. Such degradation includes irradiation hardening, creep and growth, embrittlement and crud formation. It provides a complete description of fuel rod materials, including advanced fuel designs and state-of-the-art efforts for development of accident tolerant fuels following the Fukushima accident as described in Chapter 6. This chapter covers new additives in fuels to improve performance, as well as metallic and ceramic fuels (e.g., nitrides, silicides, carbides and mixed-oxide fuels). It also describes the fuel cycle with nuclear fuel fabrication and production, operation, fuel chemistry, restructuring and fission product behaviour, thermophysical properties of the fuel, fuel performance and reprocessing of spent fuel. Severe fuel damage phenomena are covered later in the book (in Chapter 6 on reactor safety).

Chapter 5 includes a treatise on reactor thermal hydraulics to describe heat transport from the core as well as flow conditions in the reactor coolant system. It details the different flow regimes, pressure drop, critical heat flux and condensation phenomena. This discussion includes modern correlations with a broadened review of single- and two-phase flow, including viscous/inviscid, internal (pipe)/external (tank) flow, and forced and natural convective flow. This more complete treatment is important to better understand both normal operating conditions, as well as abnormal reactor operations as detailed later in Chapter 6.

Chapter 6 provides an understanding of nuclear reactor safety and reactor behaviour under abnormal conditions. It includes a discussion on reactor licensing and regulation as adopted in various countries. The chapter also discusses engineered safety features with emergency core coolant systems and reactor containment, various civil and military reactor accidents, radioactive plume migration and radiation dose analysis, nuclear emergency response and severe core damage phenomena.

Chapter 7 provides a description of the interaction of radiation with matter and fundamental concepts of health physics and radiation protection. Moreover, this chapter includes radiation dose assessment, the biological effects of radiation as well as radiation contamination treatment. It also describes the evolving subject of space-radiation protection.